

Reactor Core Design principles

AGR and HTR

B J Marsden

Nuclear Graphite Research Group

School of Mechanical, Aerospace and Civil
Engineering, The University of Manchester.

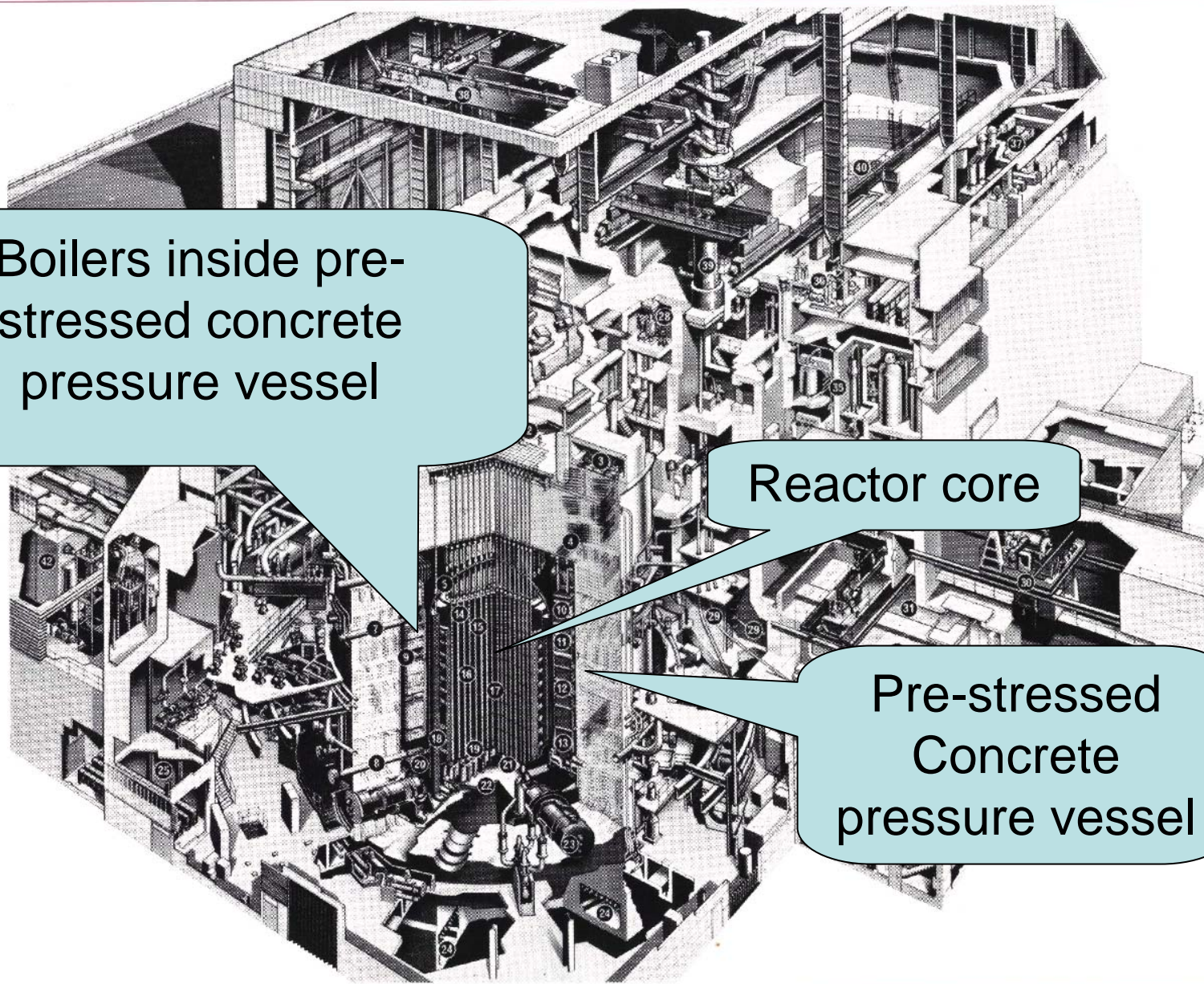
AGRs (two reactors at each site, with the exception of the prototype WAGR)

Station	MW(t) per reactor	Channels	T1 °C	T2°C	Criticality	Graphite	Designer/Builder
Windscale Advance Gas-cooled Reactor WAGR	100	253	250-325	500-575	1962	PGA	UKAEA
Hinkley Point B	1493	308	292	645	1967	AGL	The Nuclear Power Group
Hunterston B	1496	308	318	649	1977	AGL	The Nuclear Power Group
Dungeness B	1485	465	320	675	1984	AGL	Atomic Power Construction Ltd
Heysham 1	1500	324	287	651	1984	BAEL	Babcock, English Electric Nuclear
Hartlepool	1500	324	286	675	1985	BAEL	Babcock, English Electric Nuclear
Heysham 2	1650	332	292	635	1988	UCAR	National Nuclear Company
Torness	1650	332	298	635	1989	UCAR	National Nuclear Company

AGR Reactors

- All graphite moderated
- CO₂ cooled
- Electricity generated by an indirect steam cycle
- Higher outlet gas temperature – improved thermal efficiency (~42%) compared to ~30% for the Magnox
- Pre-stressed concrete pressure vessels
- Boilers inside pressure vessel
- Power density ~2.7MWs/m³ compared to ~0.7MWs/m³ for the Magnox
- Uranium dioxide fuel in stainless-steel ‘pins’ – 36 pins per element;
- Approx 308 large channels instead of huge number of small ones (over 6000 in a Wylfa reactor) in Magnox design;
- ‘Re-entrant flow’ system keeps graphite core temperature below 450C avoiding any thermal oxidation in CO₂;
- Radiolytic oxidation continues, and needs inhibitor added to coolant (methane, together with its decomposition produces hydrogen and water, are all inhibitors along with product carbon monoxide; note ‘methane holes’ in the graphite)

Hinkley Point B

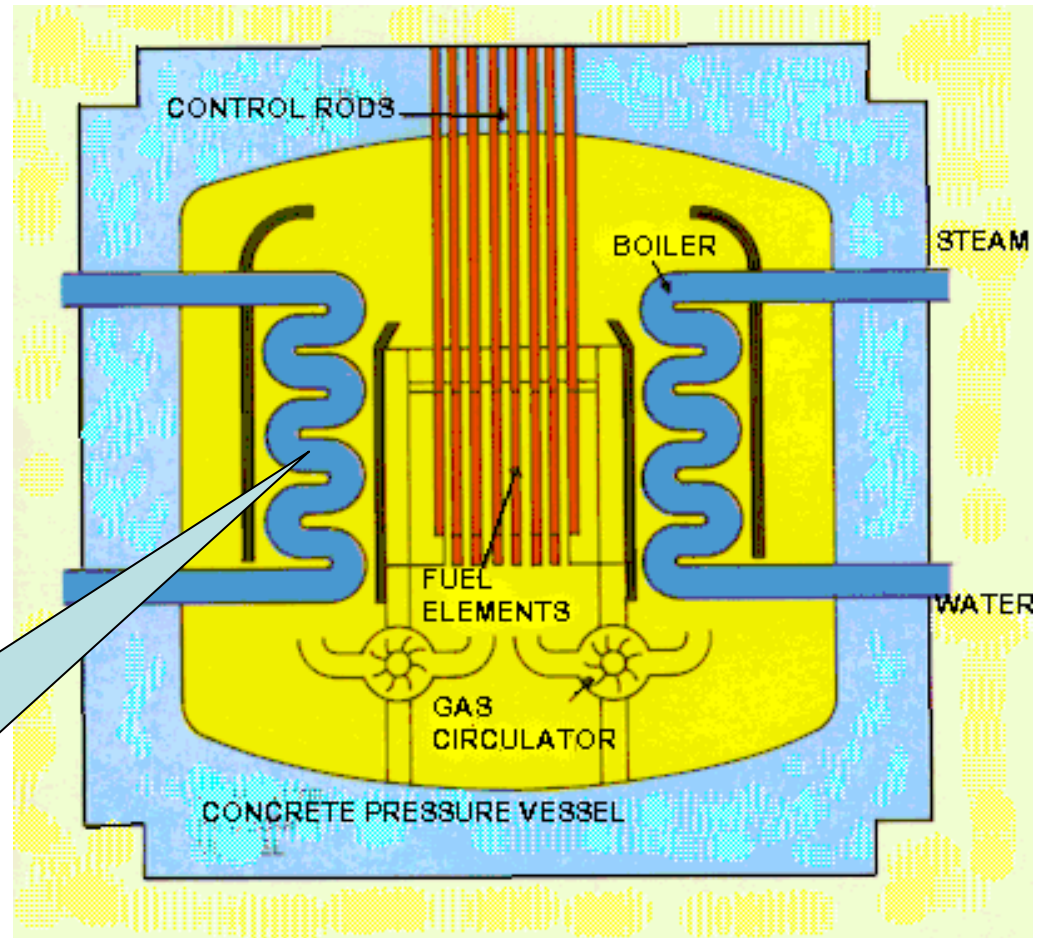


Boilers inside pre-stressed concrete pressure vessel

Reactor core

Pre-stressed
Concrete
pressure vessel

Schematic View of Advanced Gas Cooled Reactor



Boilers inside pre-stressed concrete pressure vessels

Cross Section of AGR core

Stand pipes

Charge tube

Upper shield

Side shield

DiaGrid

STANDPIPES

Fig. 1 Dungeness primary circuit arrangement

PRESSURE BELL

GAS THROTTLE PLATES

INSULATION

Graphite core

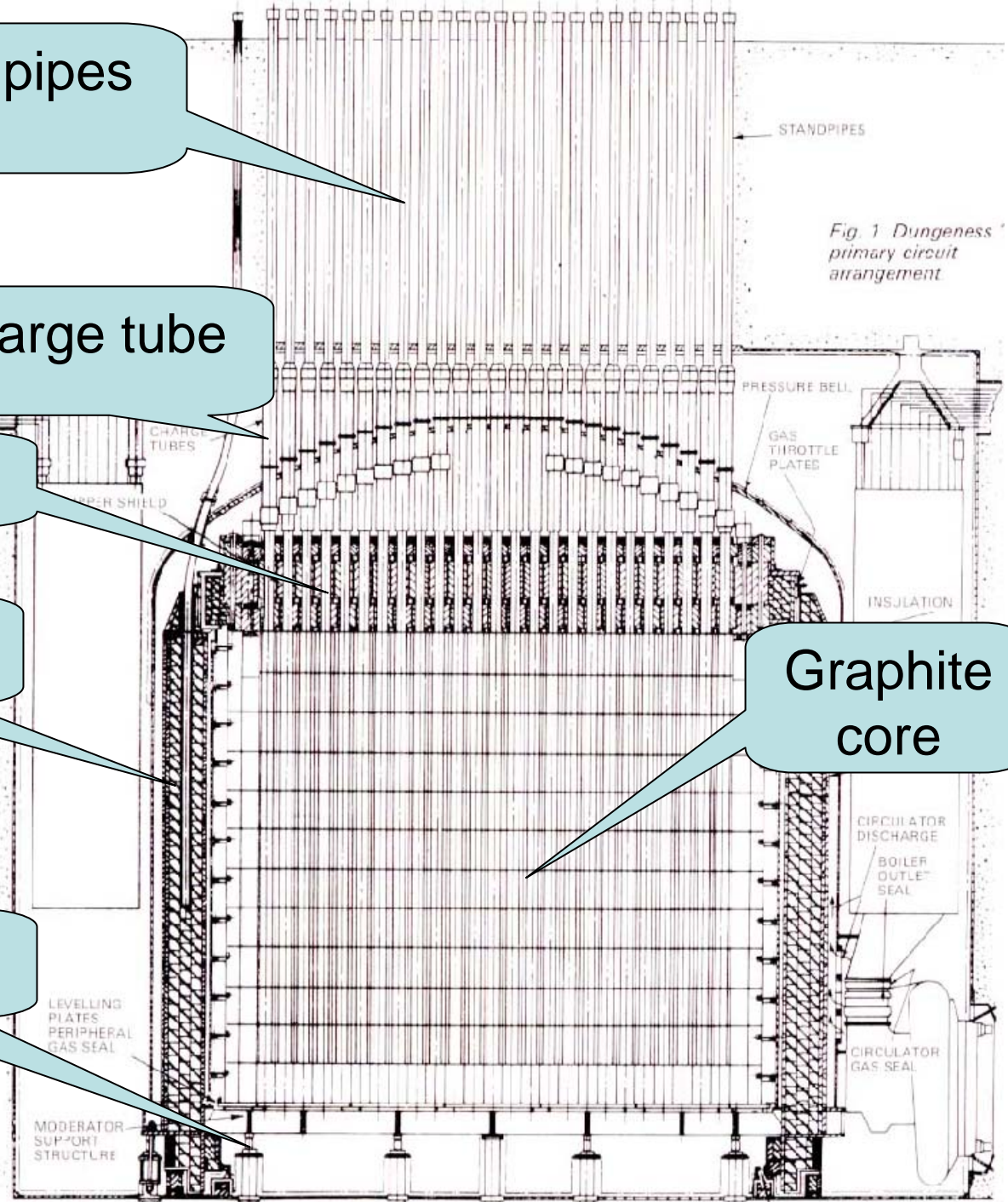
CIRCULATOR DISCHARGE

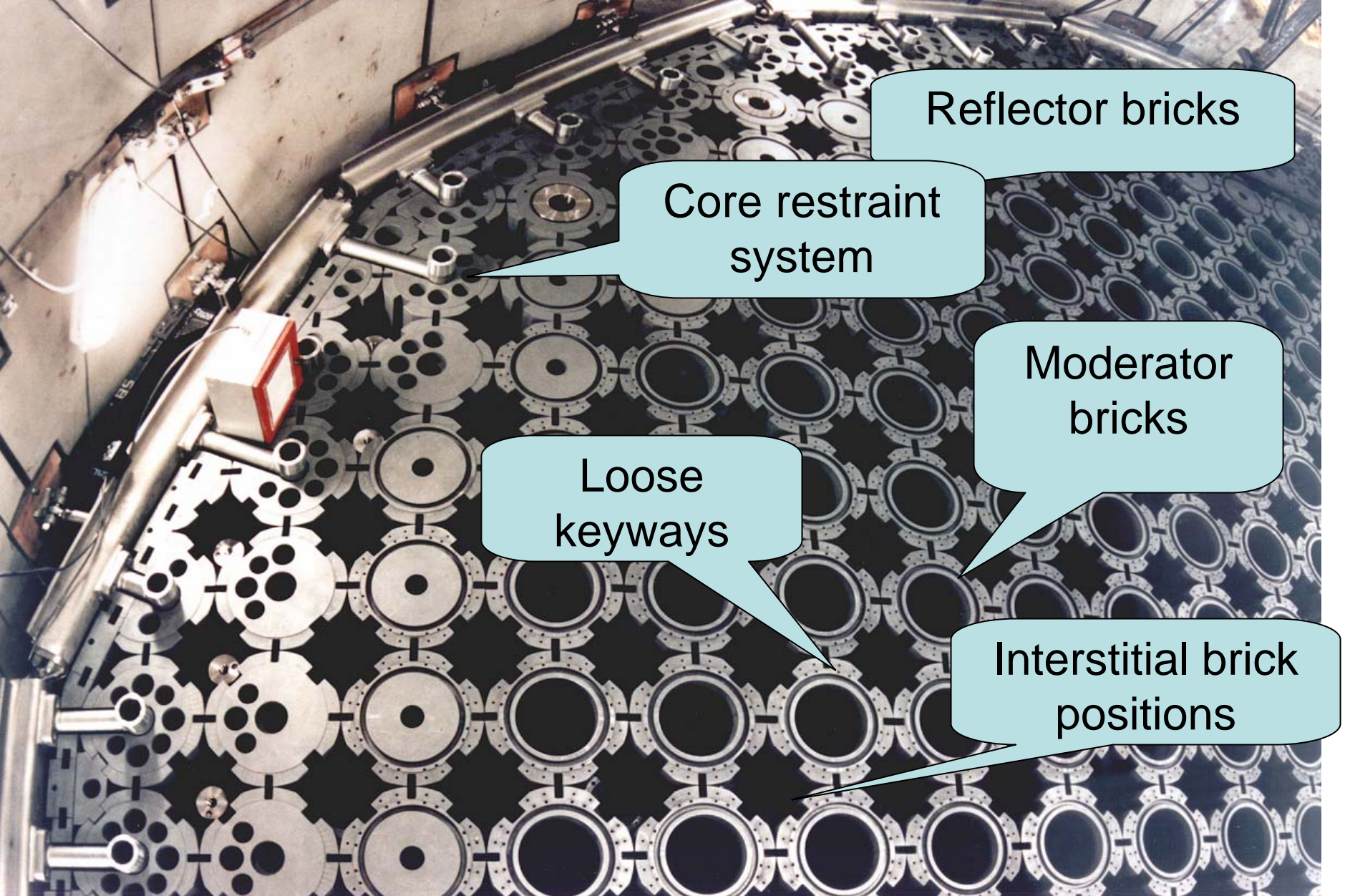
BOILER OUTLET SEAL

CIRCULATOR GAS SEAL

LEVELLING PLATES
PERIPHERAL GAS SEAL

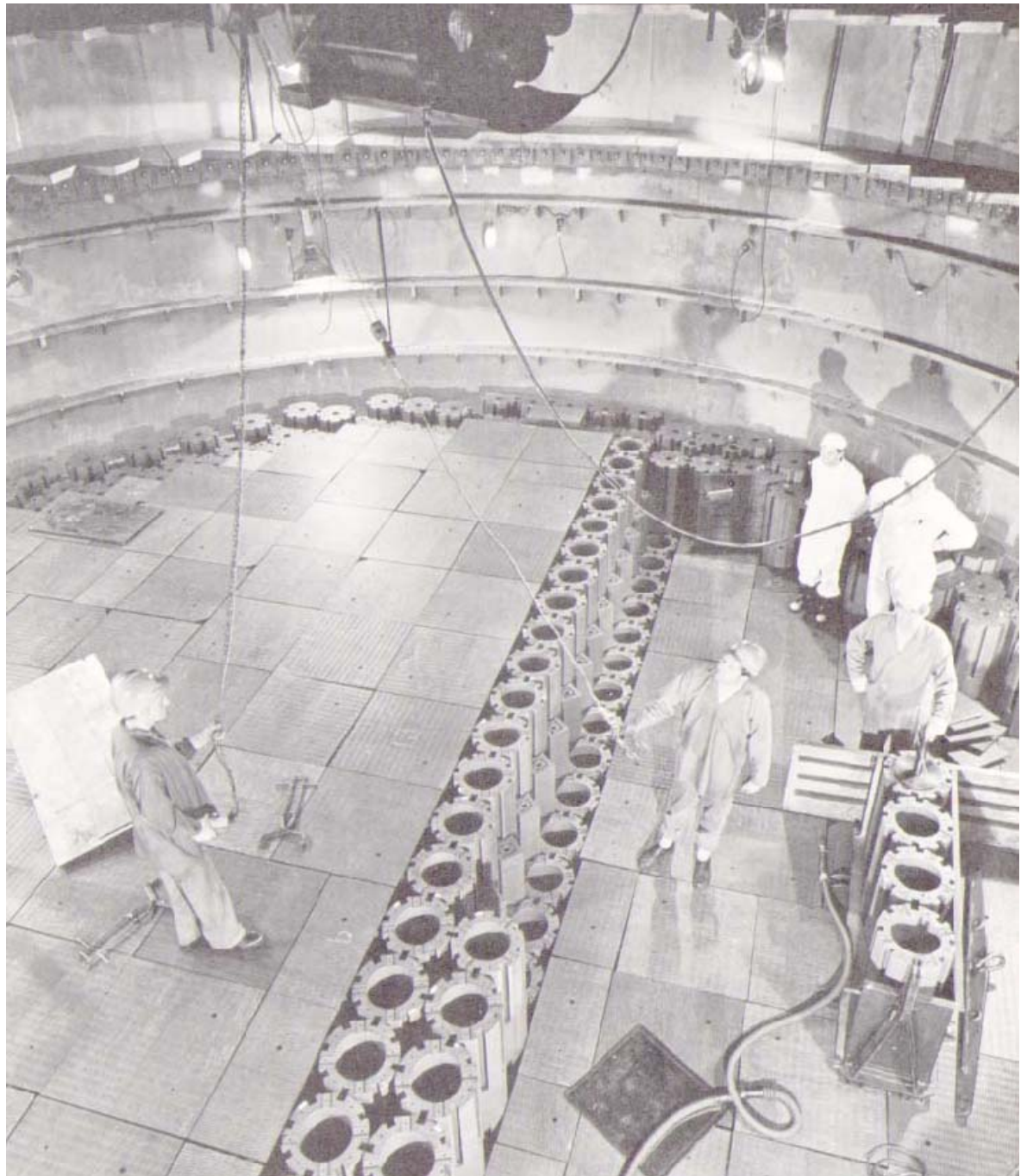
MODERATOR SUPPORT STRUCTURE





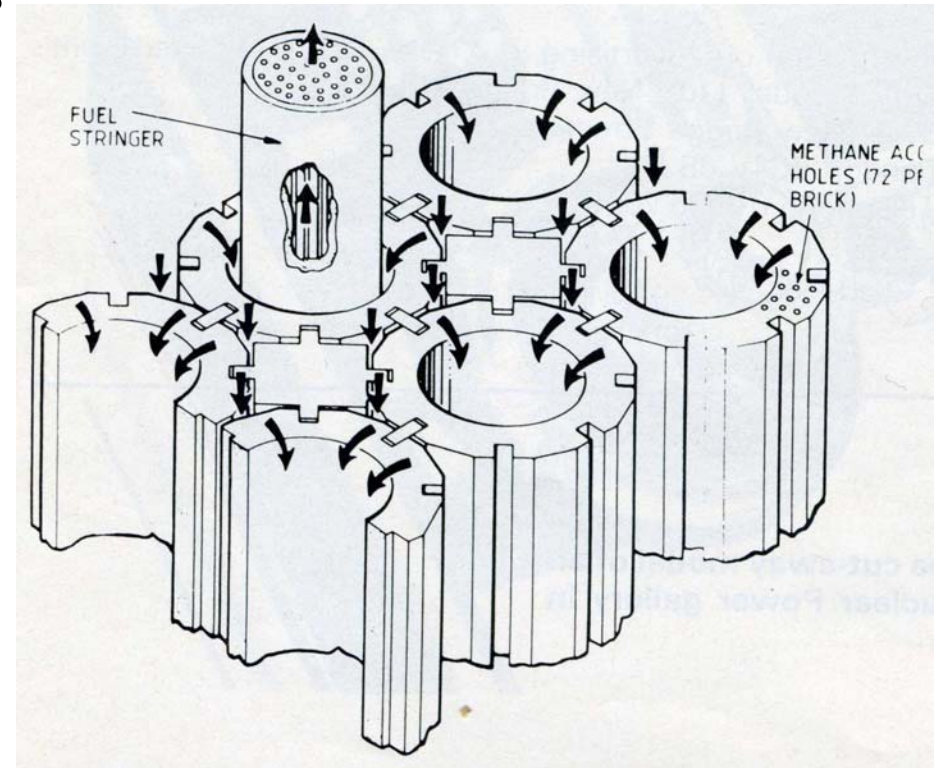
Construction of Torness Core

AGR Core Assembly

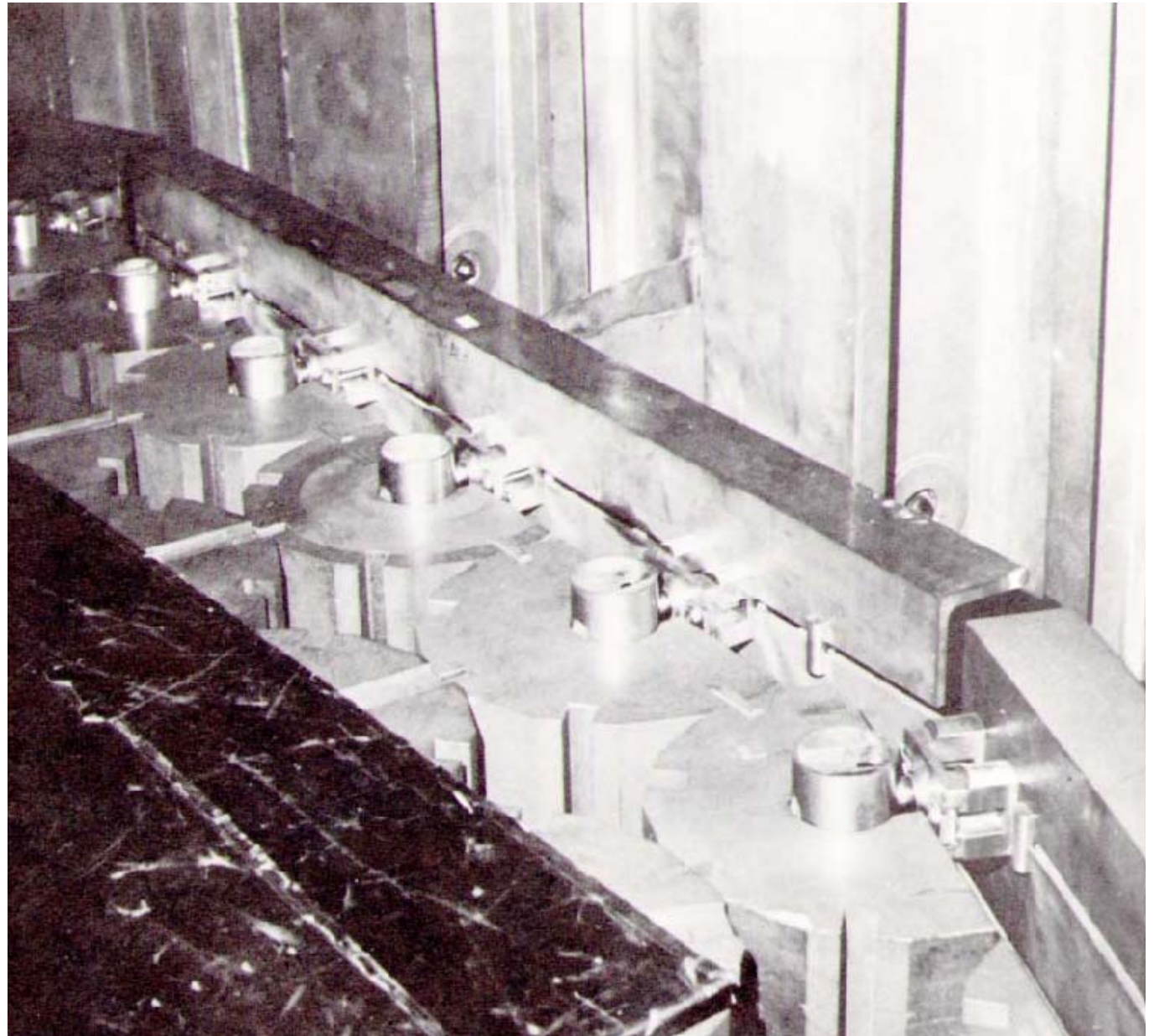


AGR Brick Assembly Showing Flow Directions

- The main coolant flow is upwards along the fuel sleeves around the fuel pin bungs
- Reverse interstitial flow:
 - between fuel stringer and brick
 - down arrow head passages
- Cross brick pressure drop in most cases (by design or accident)
- Methane holes provided to try and give access to the graphite coolant throughout the brick



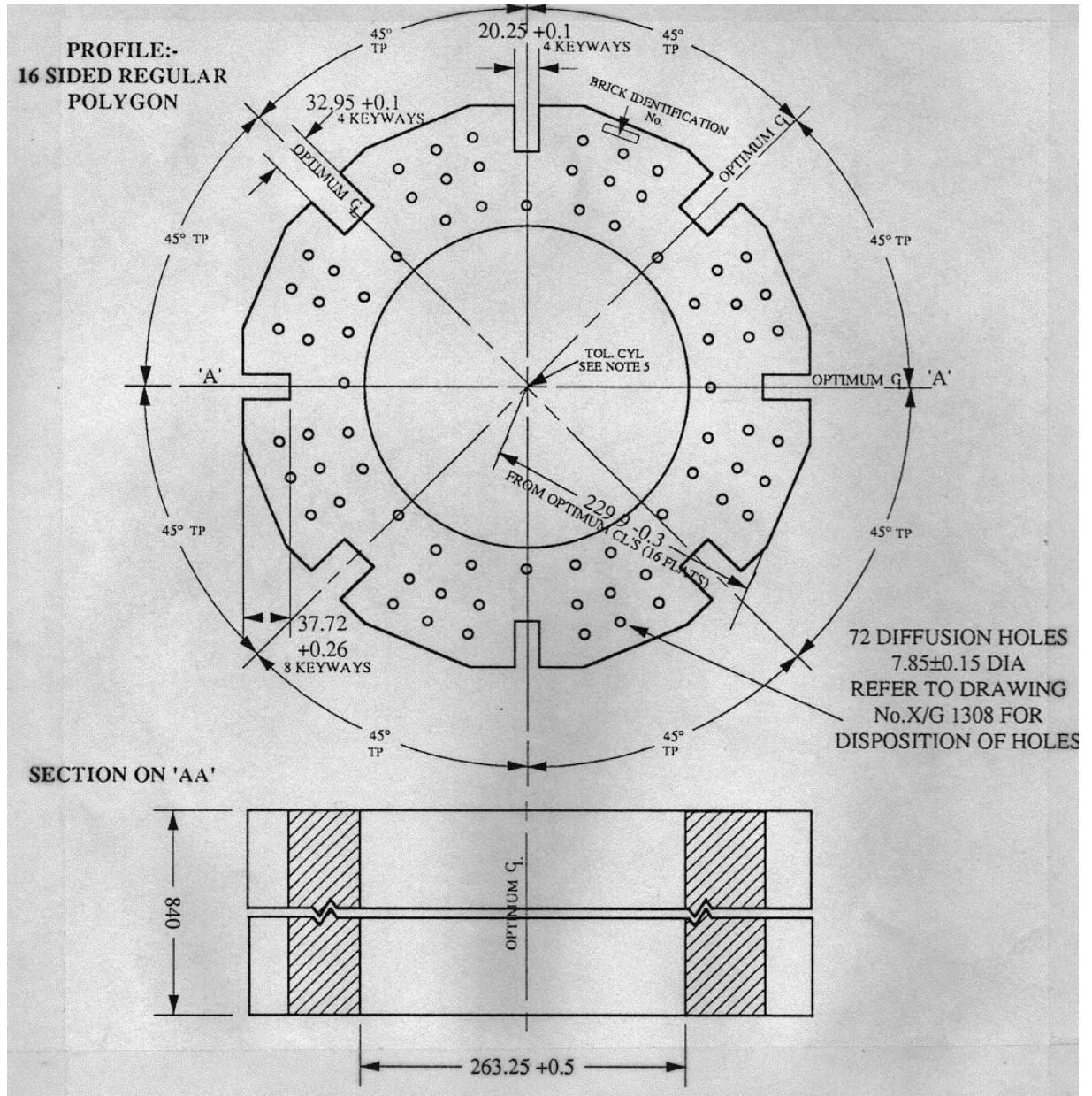
AGR Core Side Restraint



AGR Brick Design

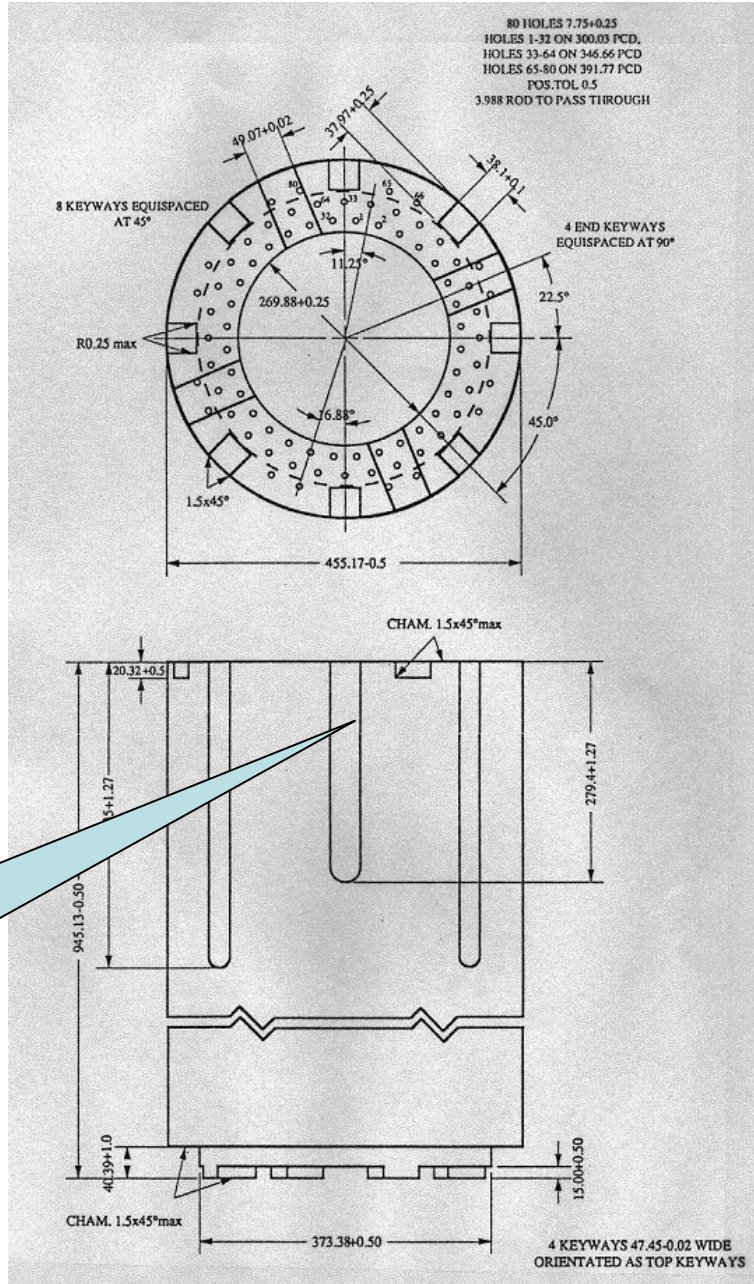
- Four different basic designs
- All include methane holes
- All have sharp keyway corners
- Some are keyed 1/3 the length
- Others keyed the whole length but with location keys over the mid 1/3 and filler keys over 2/3s the length
- Some cores include complex rocking features

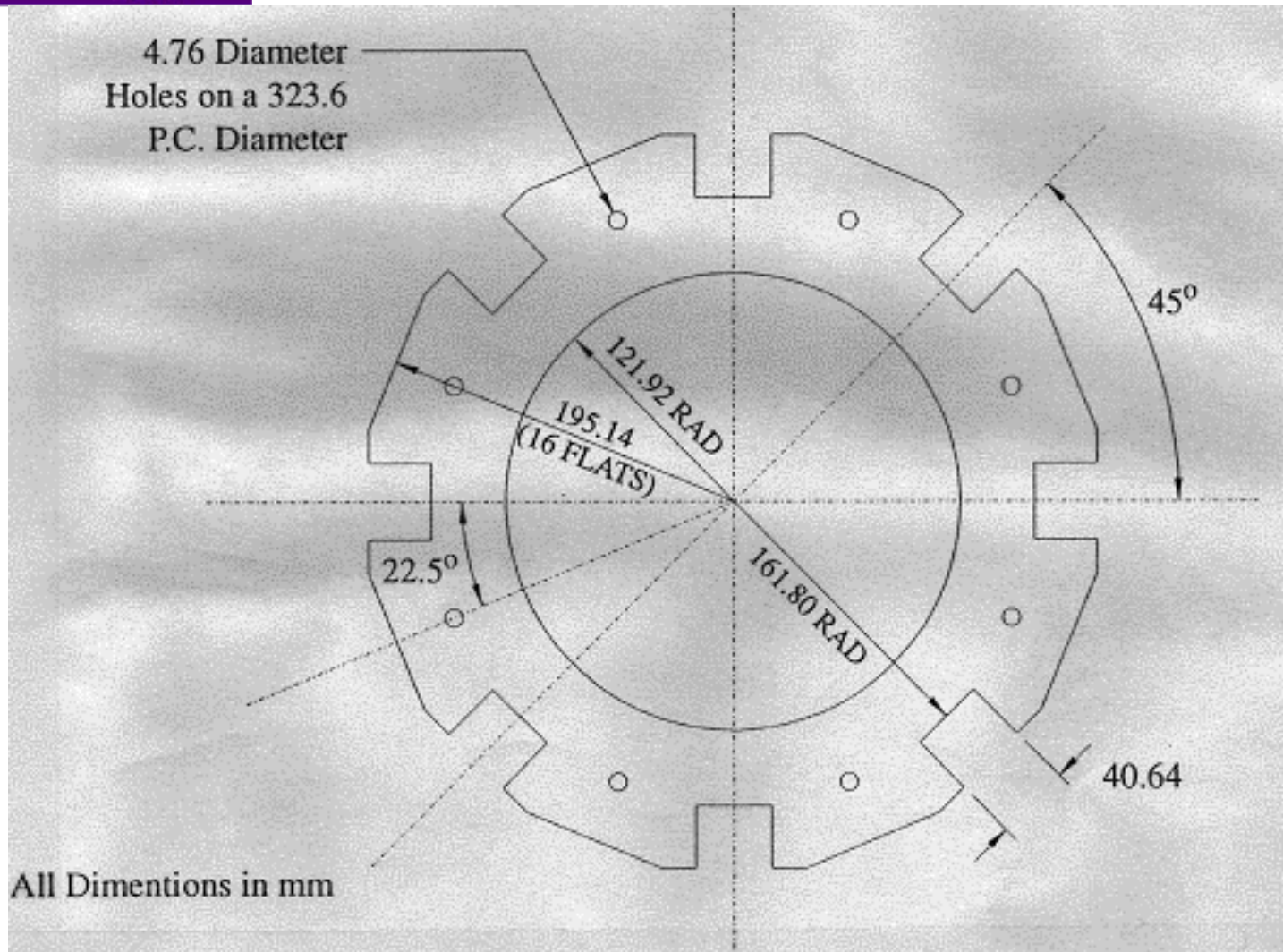
Hinkley Point B Hunterston B Fuel Brick



Hartlepool Heysham 1 Fuel Brick

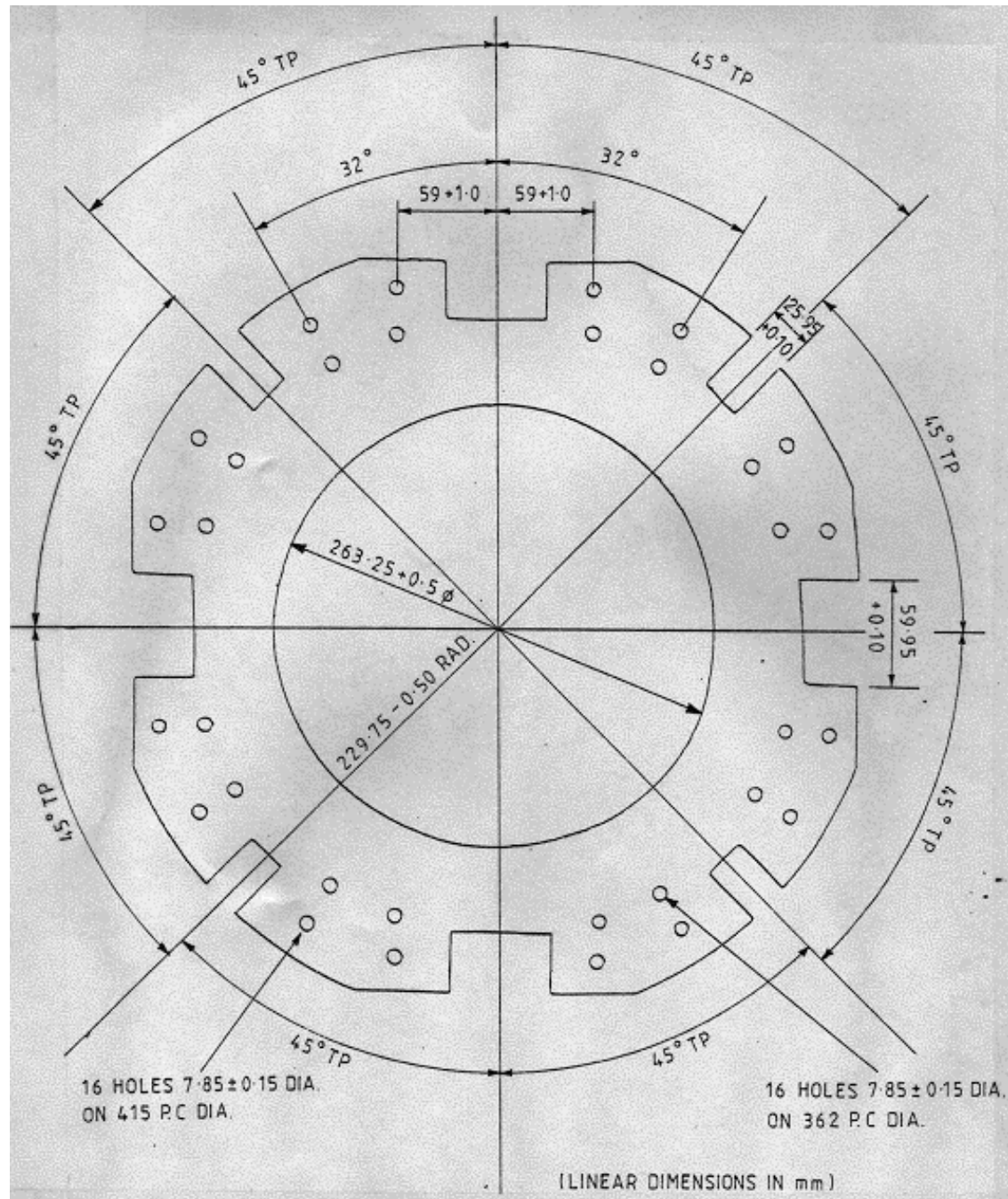
Keys only extend 1/3 the length of the brick





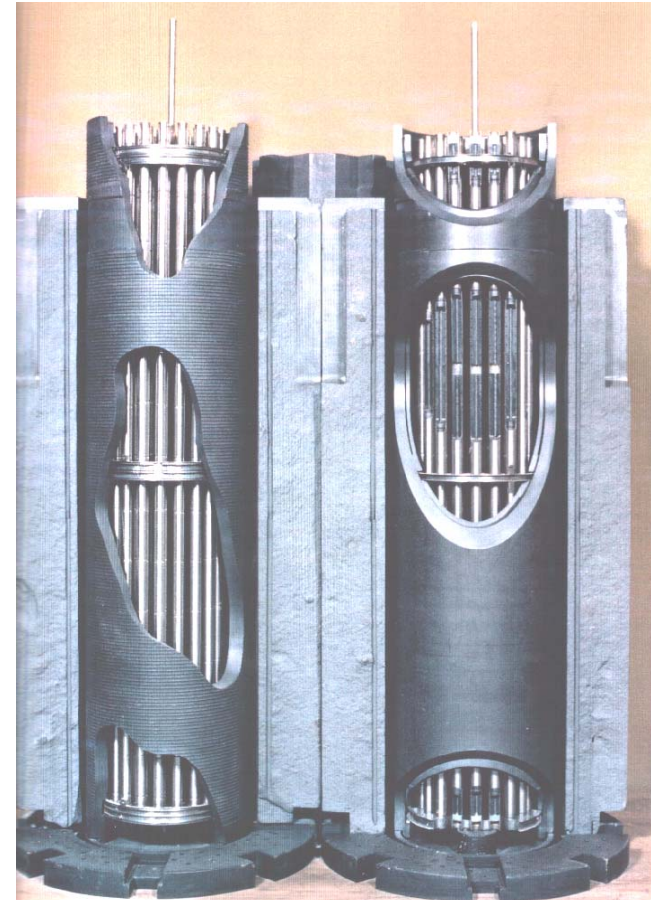
Dungeness B Fuel Brick

Heysham 2 Torness Fuel Brick



Cut away section of Hartlepool / Heysham 1 Fuel and moderator Bricks

- Eight fuel sleeves in a “stringer” suspended from tie rod (7 at Dungeness B (DNB))
- 36 pins (30 at DNB) held in 3 grids/braces
- Right double sleeve Mark 1 fuel sleeve design
- Left single sleeve Mark 2 fuel sleeve design



Safety Shut-Down Features

MAGNOX

- Control rods containing B, Cd
- Additional 'hold-down' rods
- Secondary system of 'boron balls' (actually high-B steel) – recoverable
- Boron ball system in Calder Hall / Chapelcross
- Ultimate system – blowing in 'boron dust' – actually boron oxide (Never used).

AGR

- Articulated Control rods containing B, Cd
- Additional 'hold-down' rods
- Secondary system of nitrogen injection
- Additional systems such as 'boron balls' are being considered as retrofit to satisfy the Nuclear Installations Inspectorate's concerns about core integrity and core restraint integrity

HTR Core Design

High Temperature Reactor

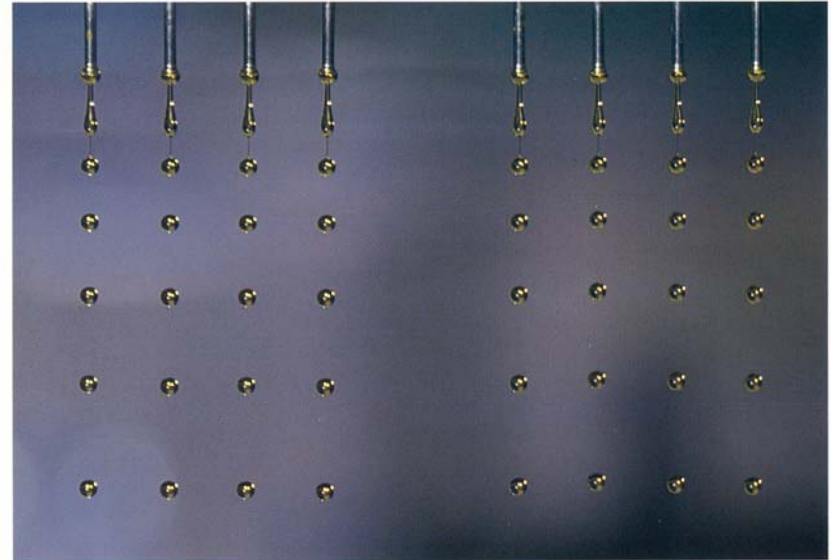
- High gas output temperature, above 700°C
- Ceramic Fuel
 - Pyro-carbon and Silicon Carbide Coating
- Graphite Reflector
- Helium Gas Cooling
- Direct Gas Turbine or Indirect steam generation
- Prism or Pebble Bed configuration

High Temperature Reactors

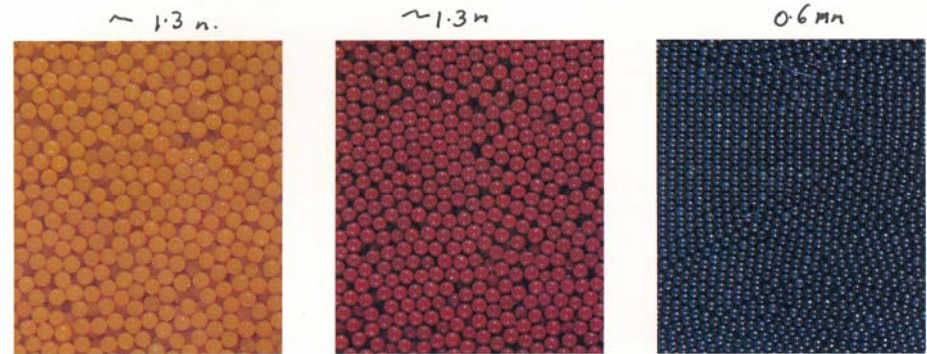
Reactor	Purpose	Country	Criticality	Shut-down	Type	MW(t)
Dragon	Research	OECD	1962	1976	Prism	20
Peach Bottom	Research	US	1966	1974	Prism	115
Fort St. Vrain	Power	US	1977	1992	Prism	842
AVR	Research	Germany	1967	1988	Pebble	49
THTR	Power	Germany	1983	1989	Pebble	750
HTTR	Research	Japan	2003	Operating	Prism	30
HTR-10	Research	China	2003	Operating	Pebble	10
HTR-PM	Prototype	China	Under Design		Pebble	two 250 units
PBMR	Power	South Africa	Under Design		Pebble	240
GT-MHR	Power	International	Design Concept		Prism	600
ACAICA	Power	NRG	Design Concept		Pebble	40
NGNP	Research	US	Under Design		Pebble or Prism	600

Production of Coated Fuel Particles

- Uranyl Nitrate droplets
- ADU Particles
- Calcined to Produce
- UO_3 Particles
- Sintering and coating to produce UO_2 Kernels



Vibrational dropping



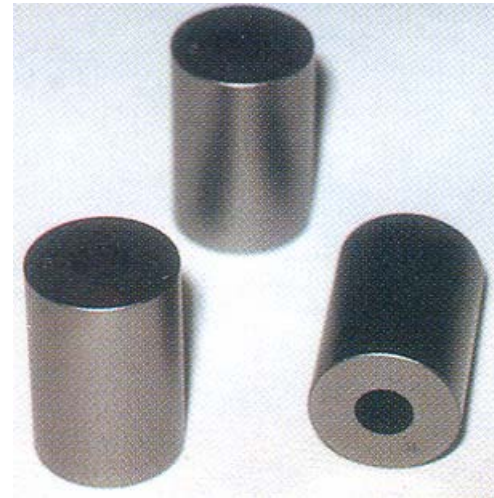
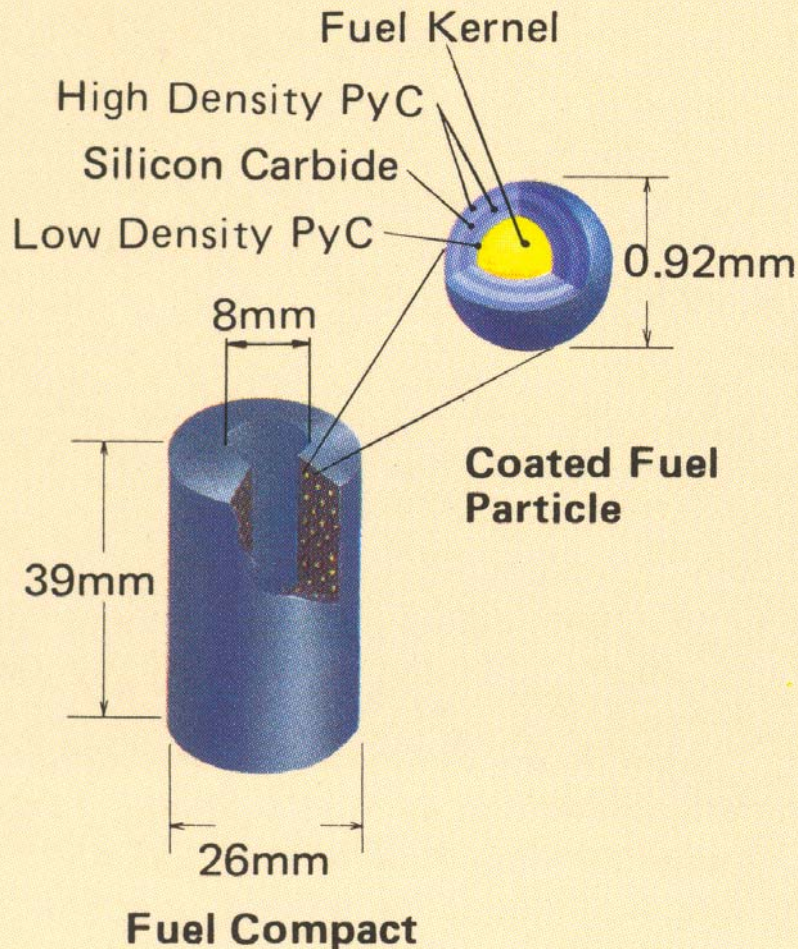
ADU particles

UO_3 particles

UO_2 kernels

HTR Prism Fuel Compact

Compacts manufactured from crushed natural and artificial graphite in formaldehyde resin (baked)

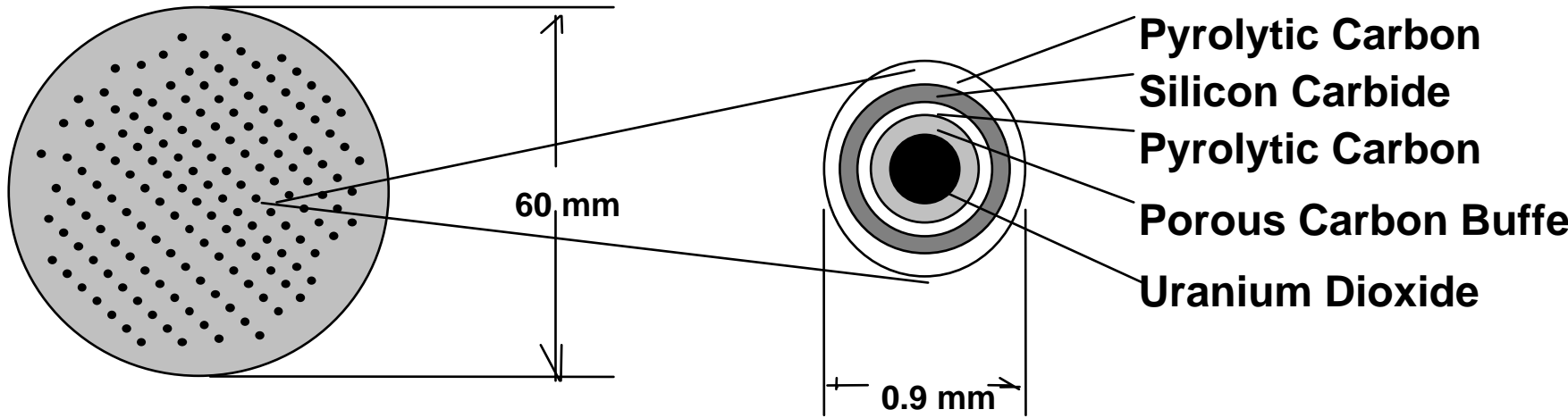


Prism
Fuel
Compact

Pebble
Fuel
60 mm
diameter



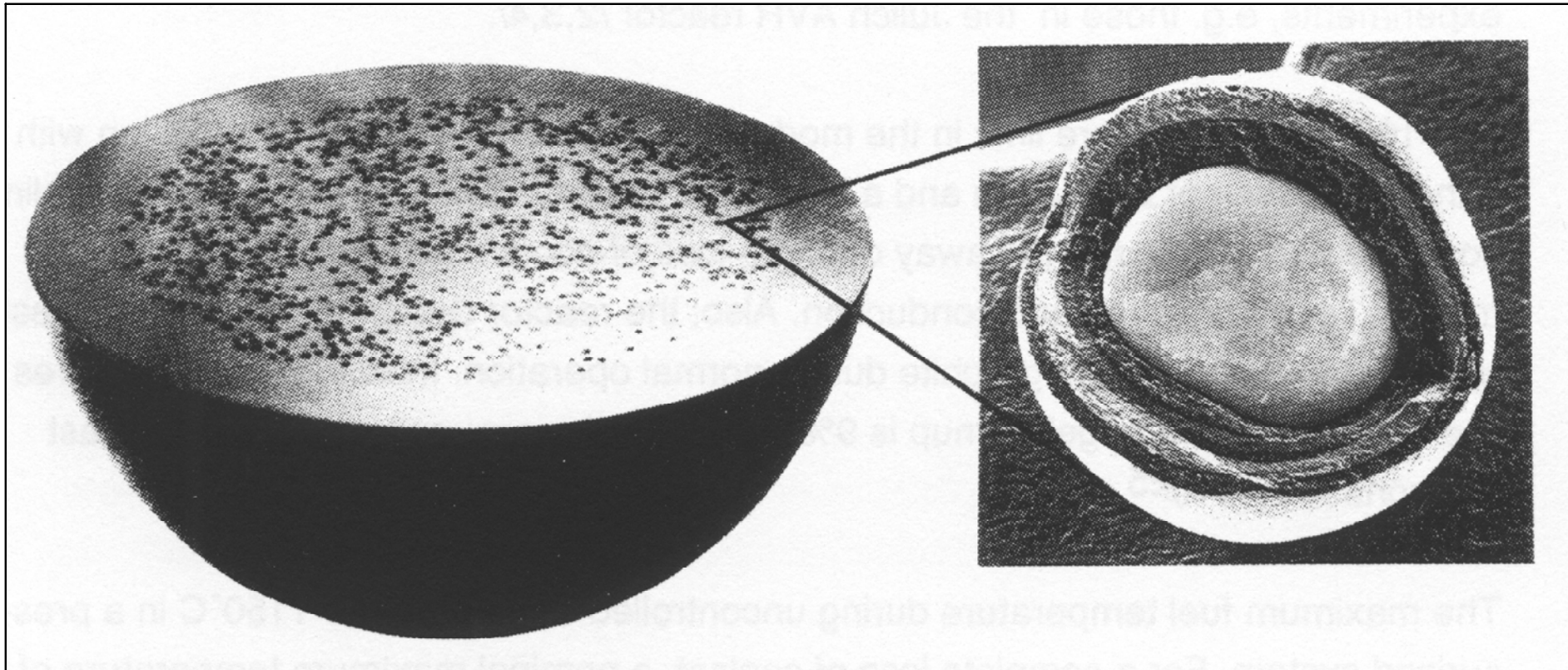
Fuel Design

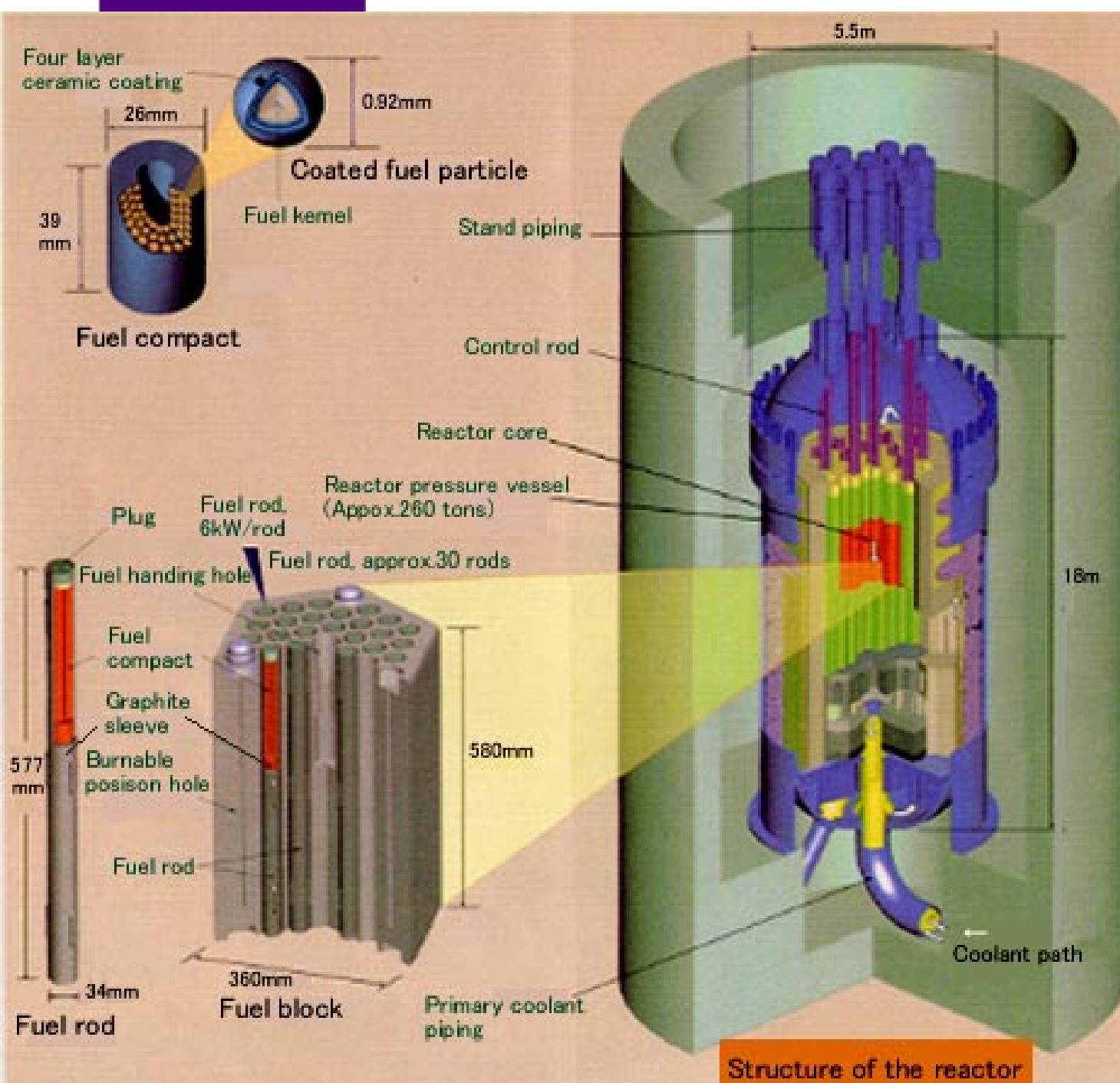


Pebble Fuel

HTR Pebble Cross-section

Cut-away Coated Particle



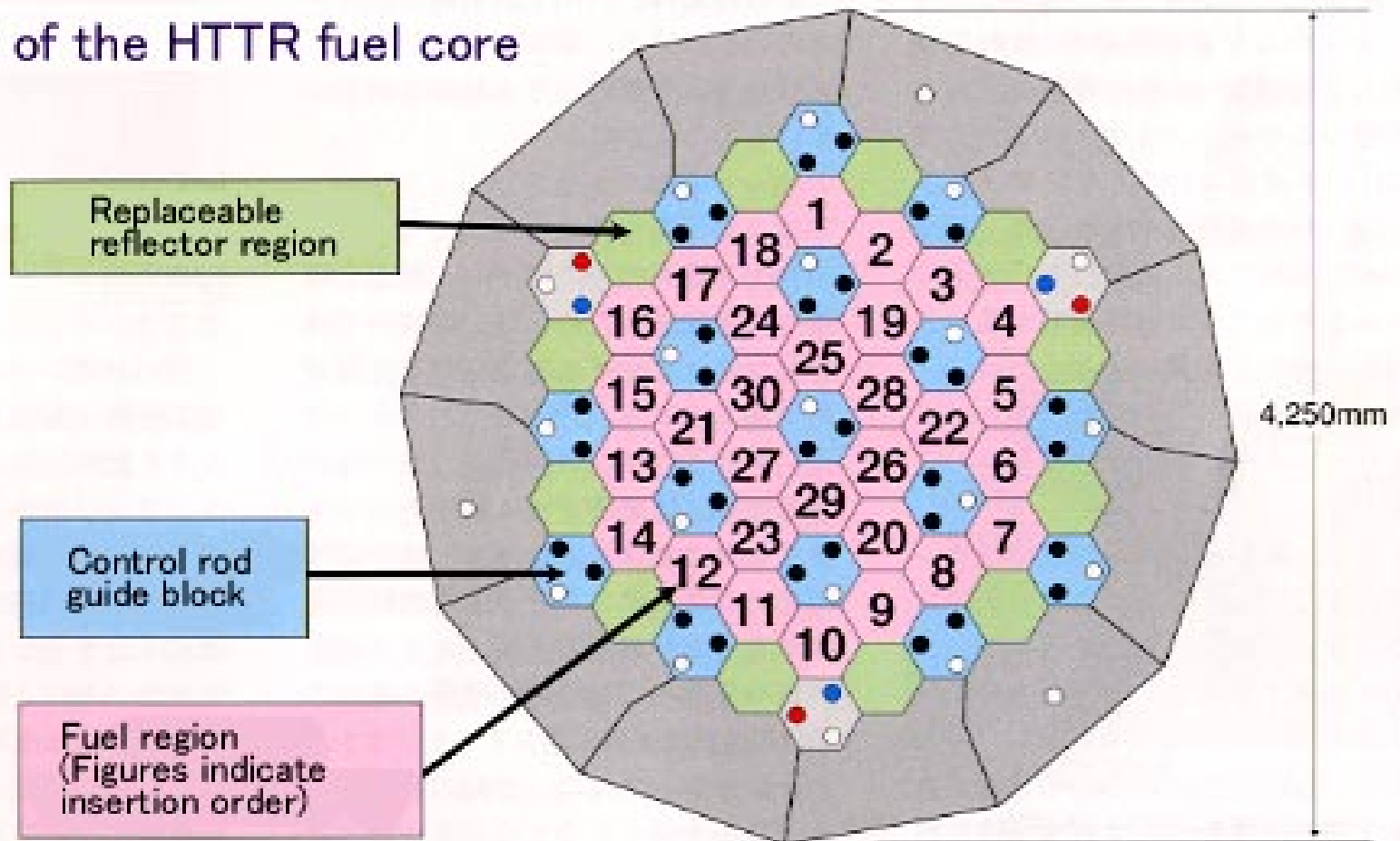


HTTR Prism Design

Structure of the reactor

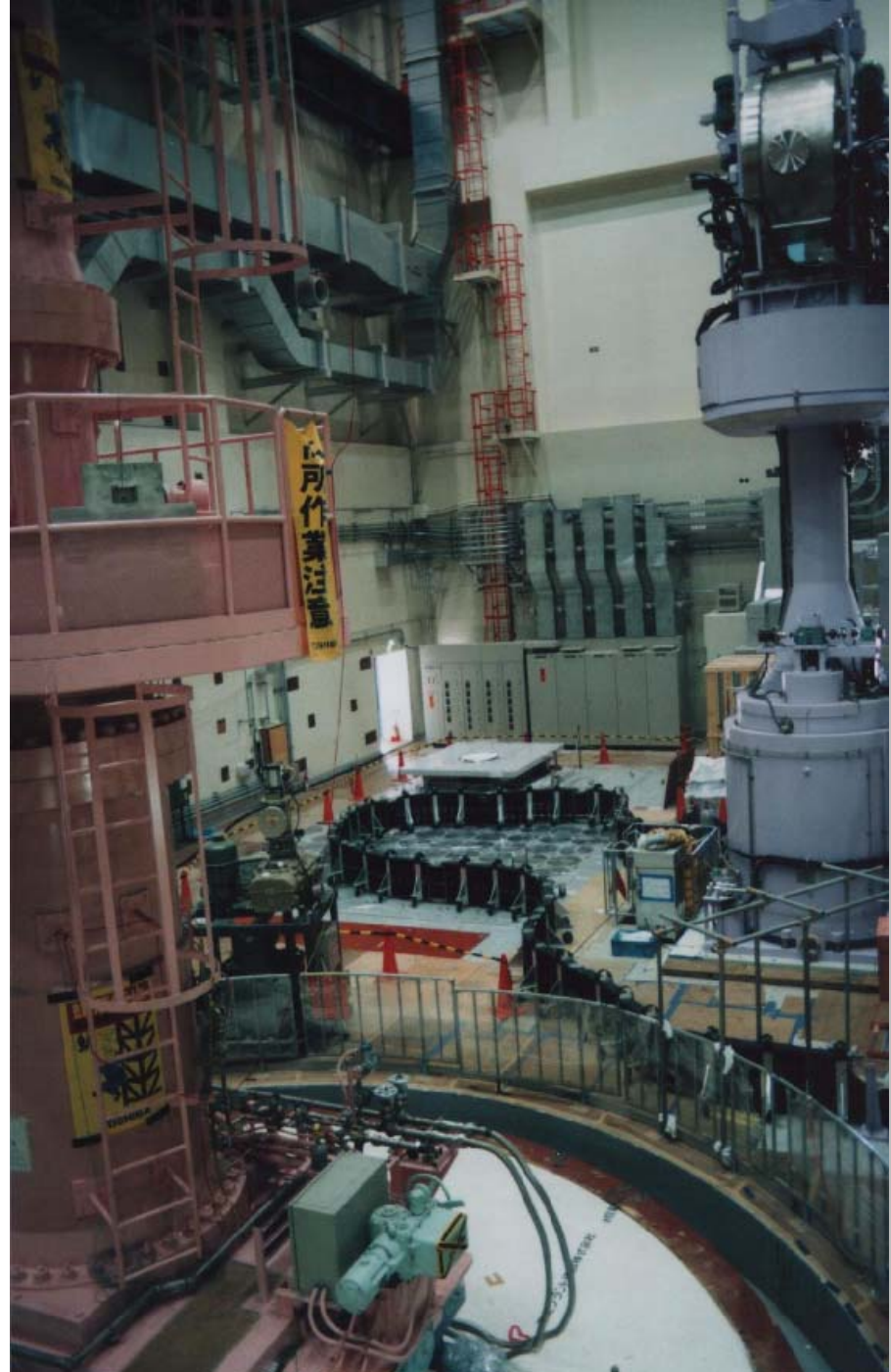
HTTR Core Cross Section Prism Design

Position of the HTTR fuel core



HTTR JAERI - OARAI





Fuel Blocks



Location pins, (boron carbide poisoning)

Fuel Pins



Fuel Pin





Control rod holes



Prototype carbon composite control rod segment



Fuel Handling Head



Horizontal
assembly

Seismic Assessment Rig

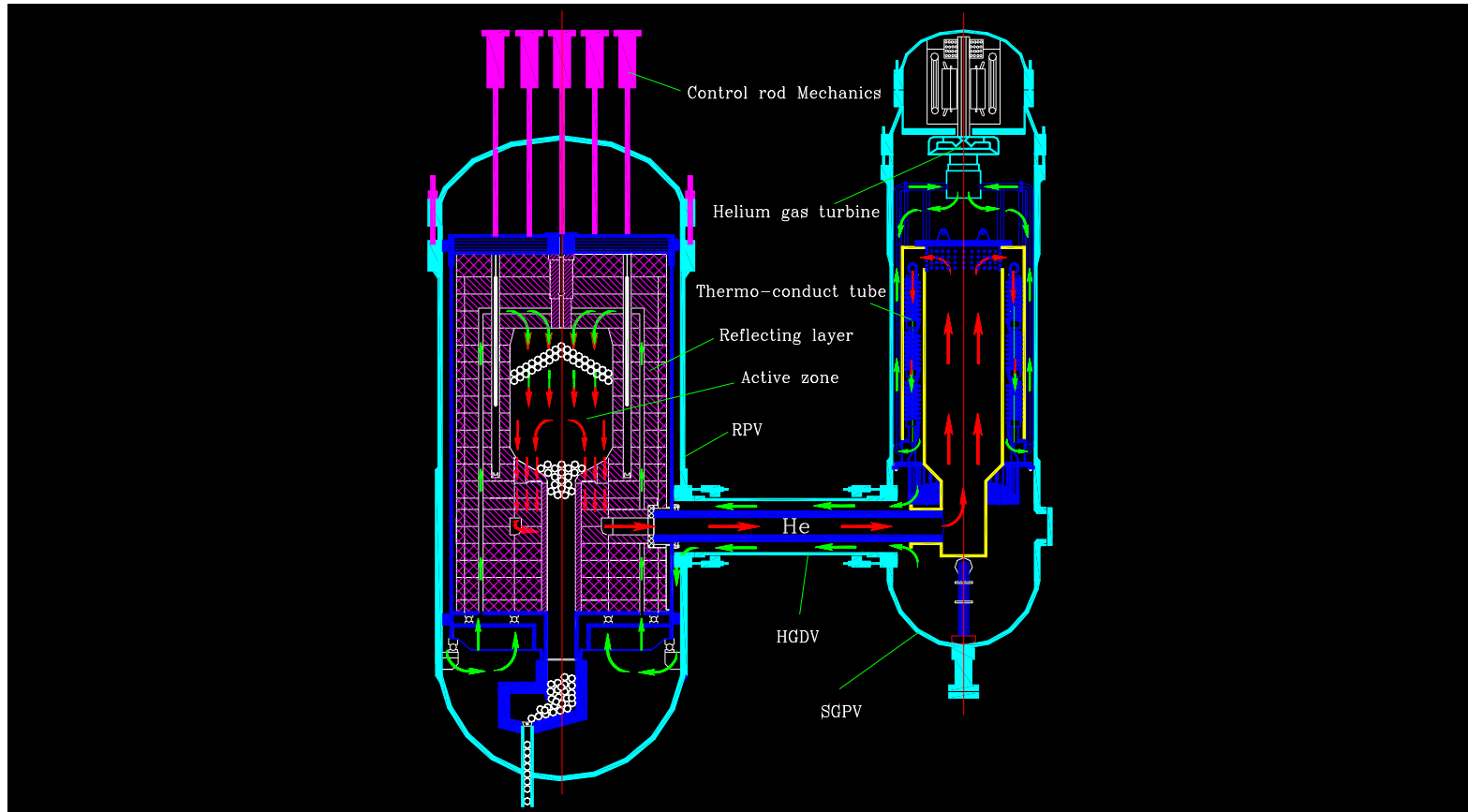
Vertical
assembly

Scale components, metal
insert to give density of fuel

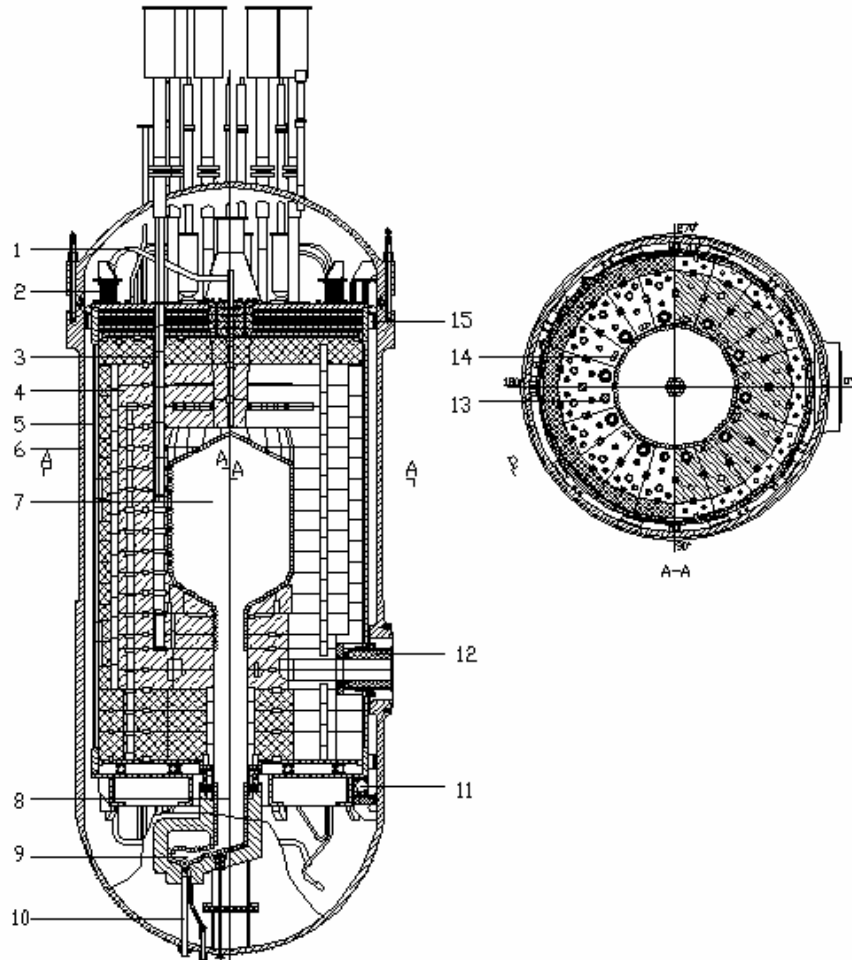
Waste Heat



Vertical Cross Section Through Reactor



Cross Section Through Reactor Core



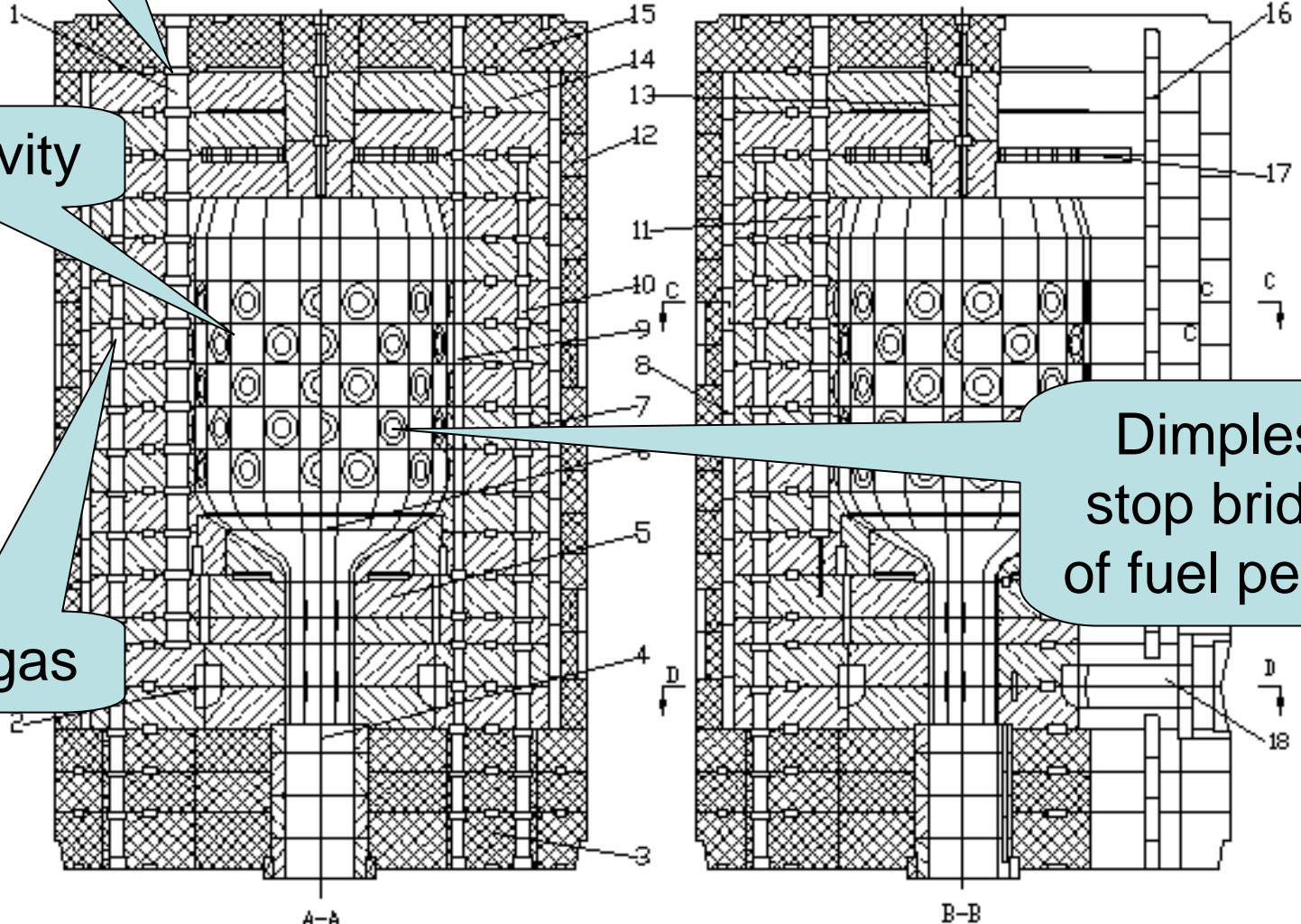
Channels

Control rod
hole

Core cavity

Return gas

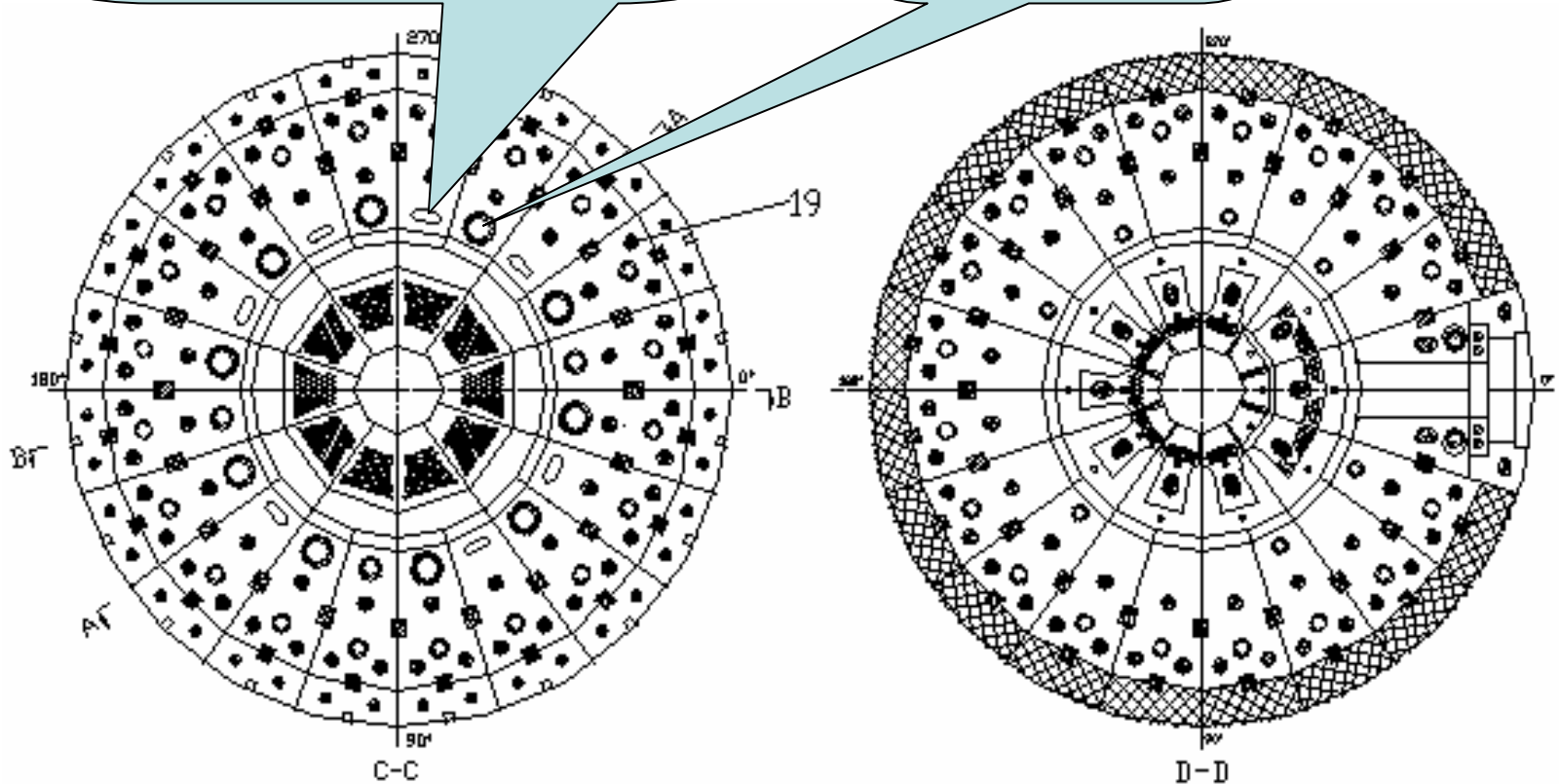
Dimples to
stop bridging
of fuel pebbles



Horizontal Cross Section

Boron ball shut down holes

Control rod hole



Construction

Control Channel
Sleeve

Vertical Side
keys

Graphite Dowel

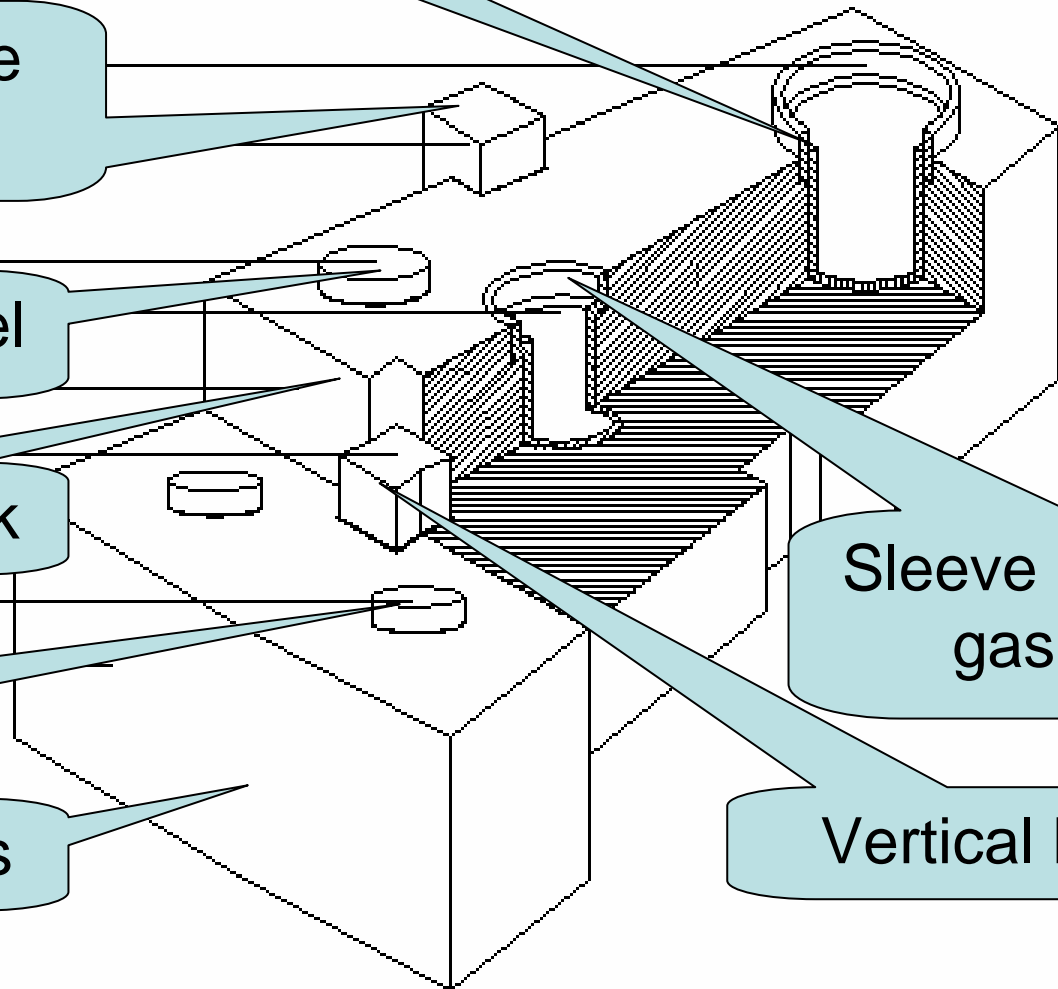
Reflector Block

Dowels

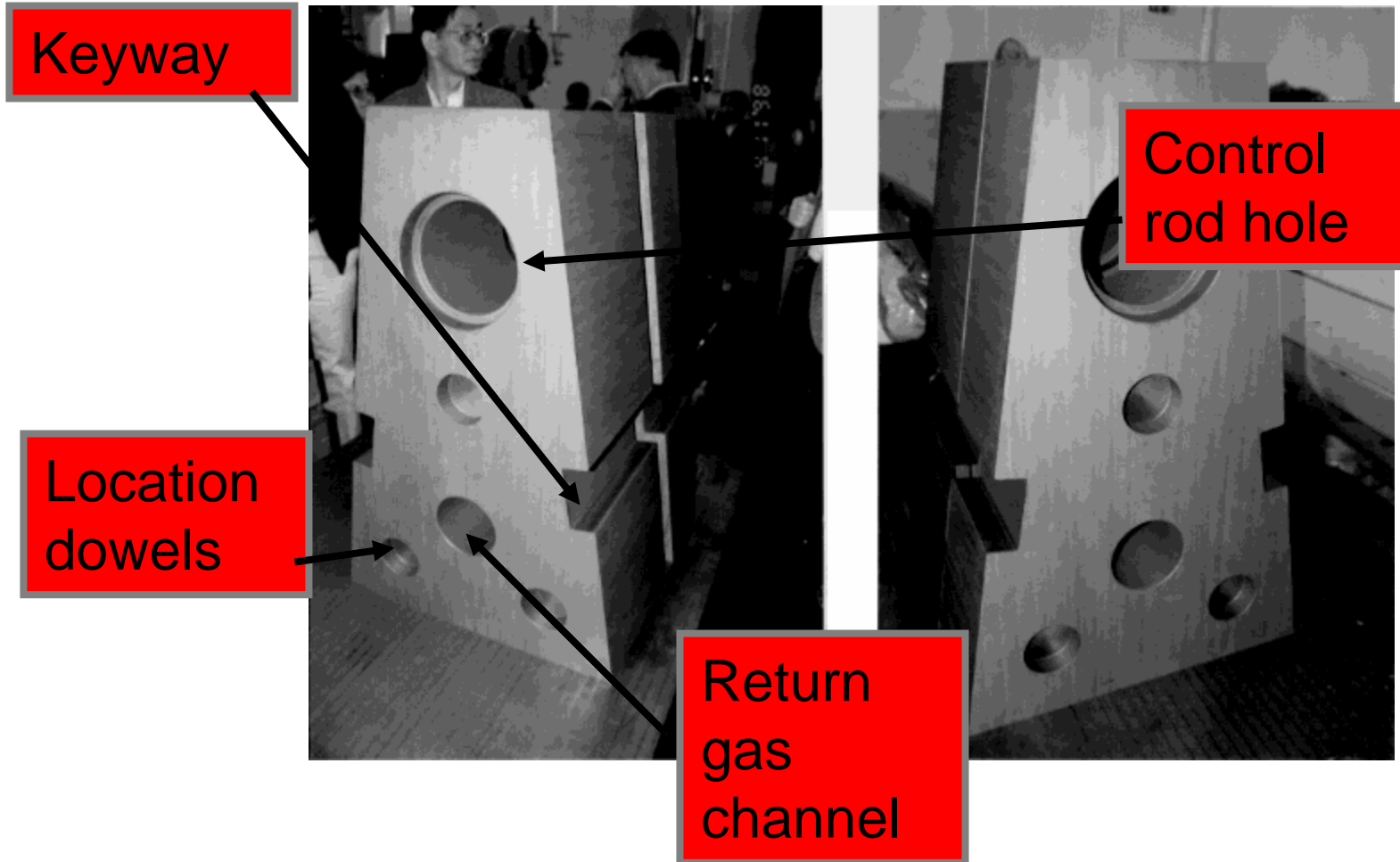
Carbon Blocks

Sleeve inside return
gas channel

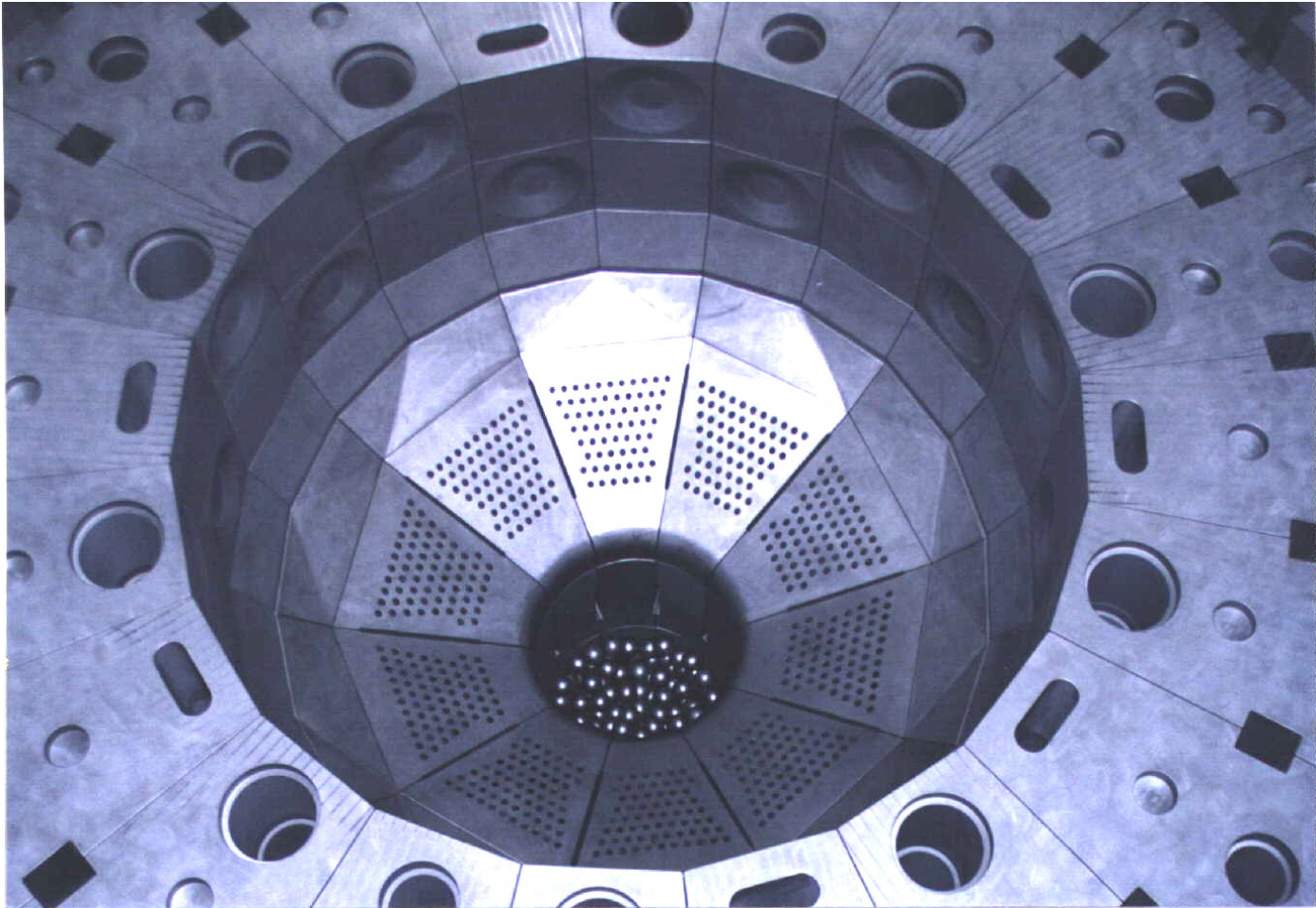
Vertical Key



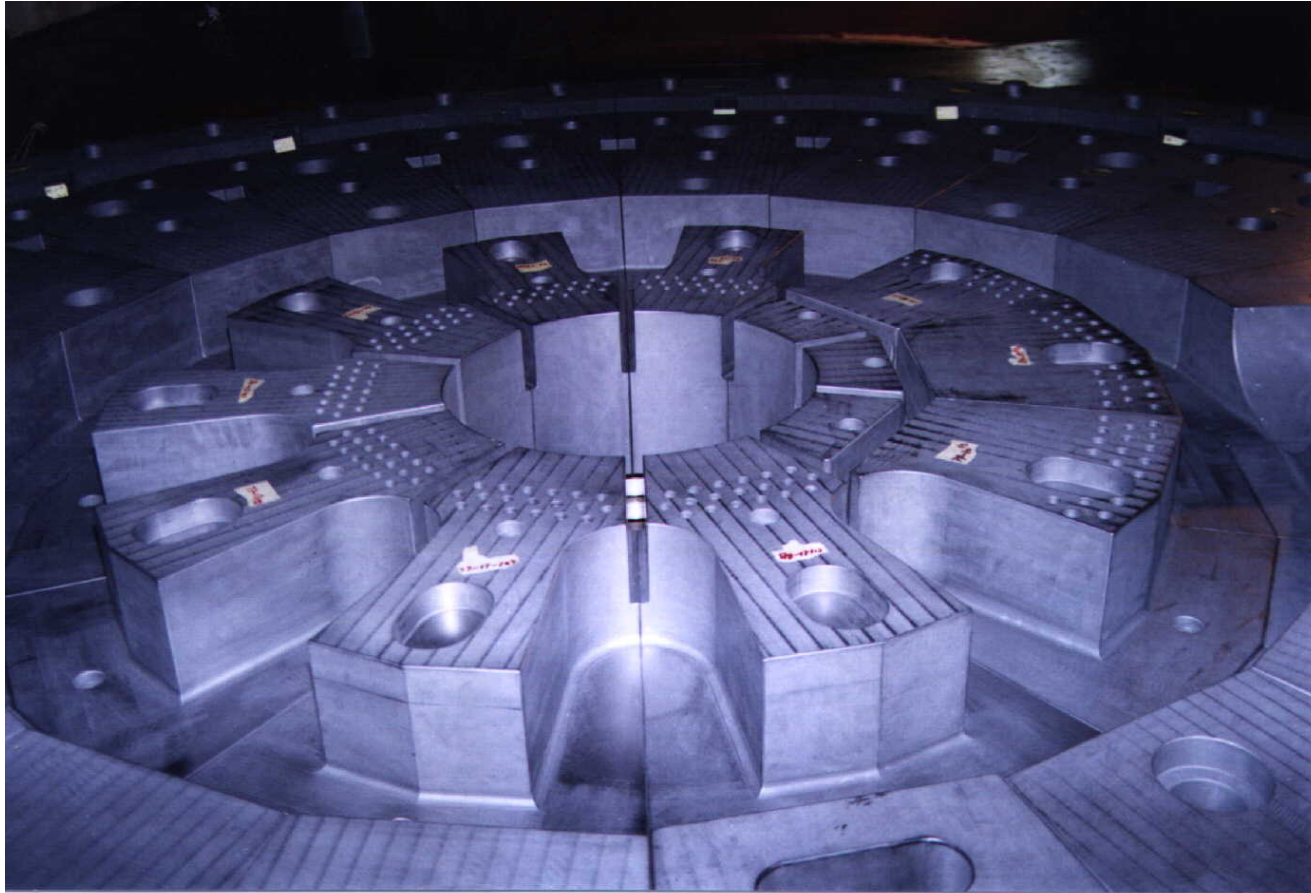
Typical HTR Graphite Reflector Blocks HTR-10



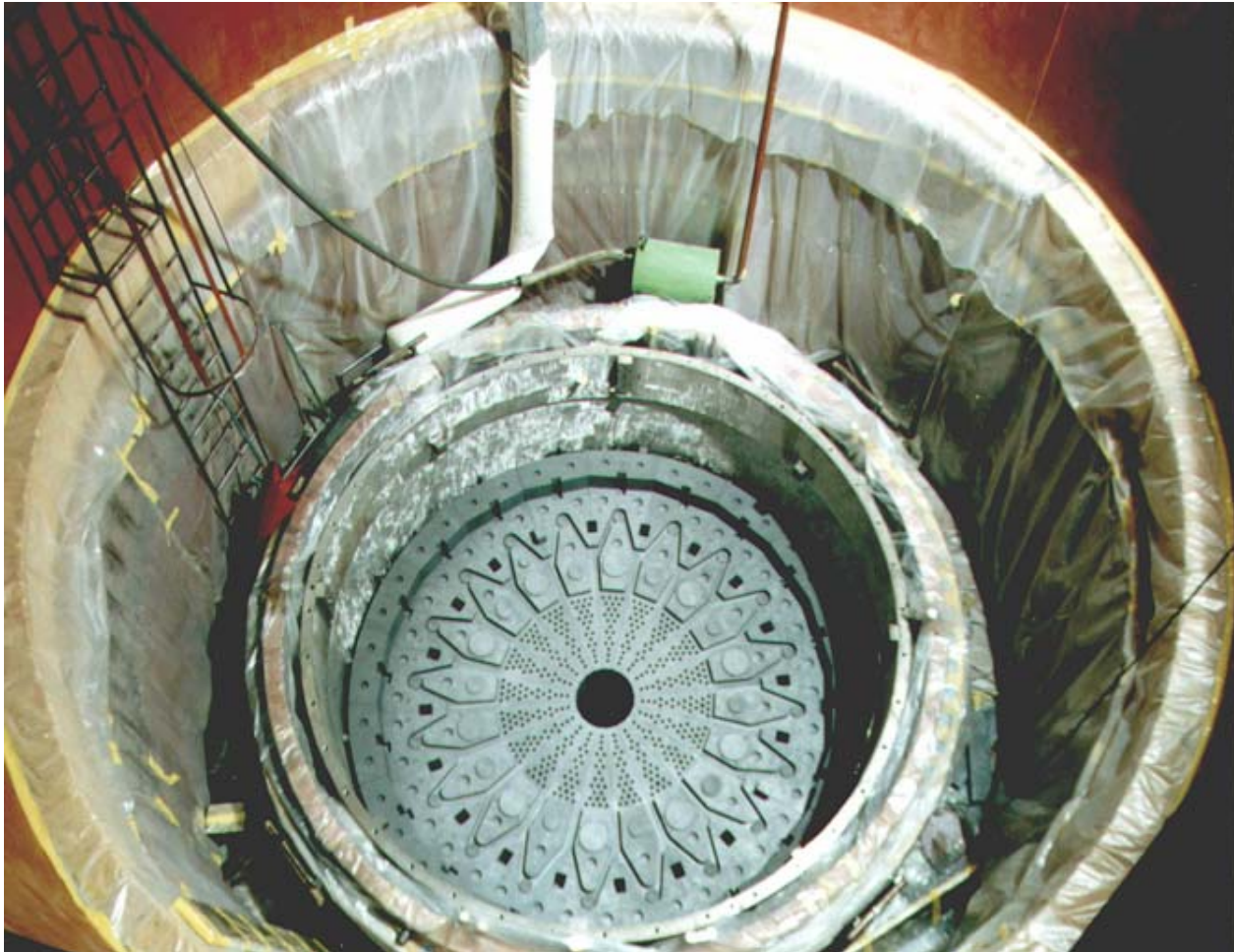
Lower Core Structure



Lower Half of Hot Gas Chamber

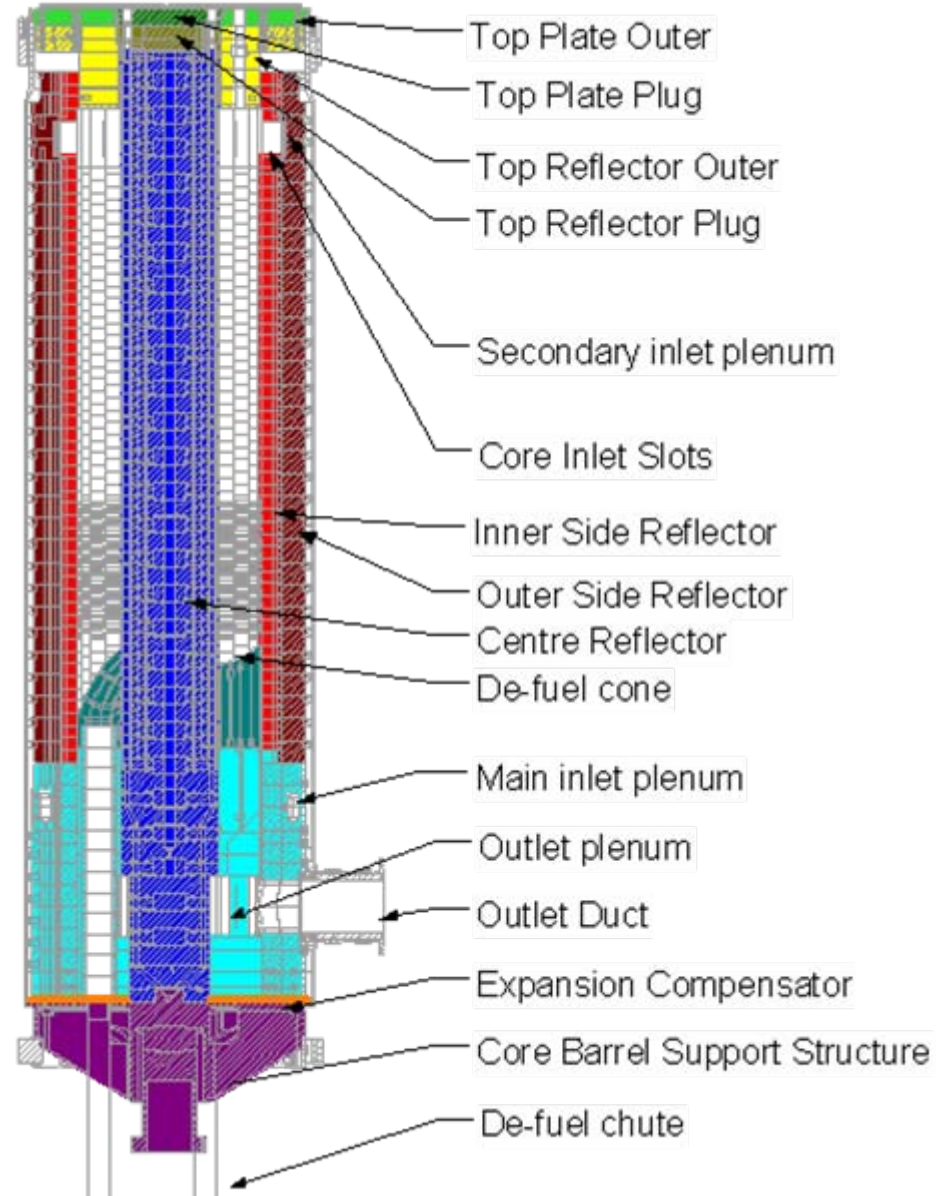


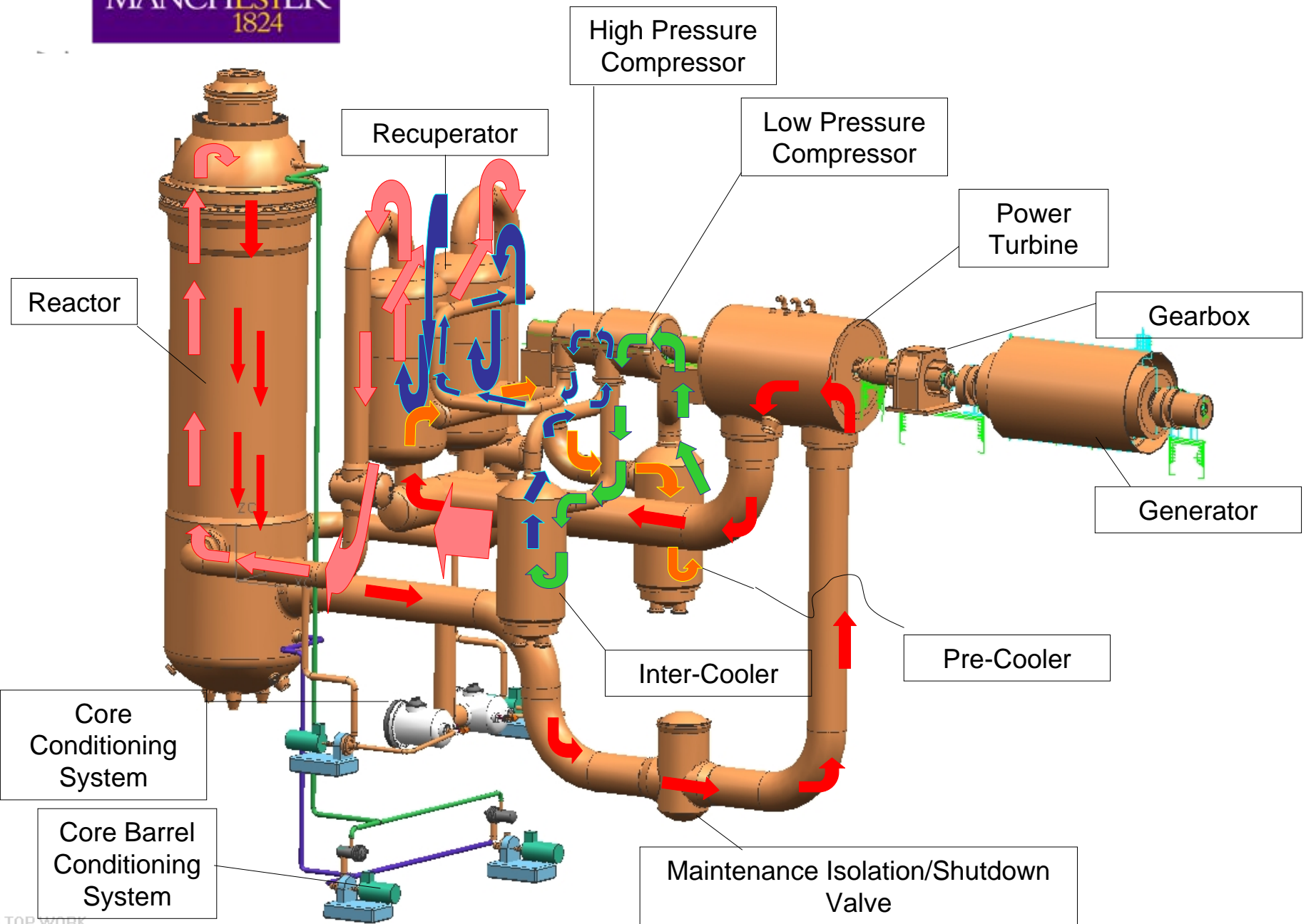
Top Reflector



PBMR Design Features (Original Concept)

- Utilises direct-cycle gas turbine
- note central graphite reflector





VHTR Migration Path:

